



ERRATUM - ICFRM-21 CONFERENCE PROGRAMME

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The International Advisory Committee

PINTSUK, Gerald, Forschungszentrum Jülich, Germany

WANG, Yugang, Peking University (PKU), Beijing, China

ZHOU, Haishan, Inst. of Plasma Physics (ASIPP, CAS), China

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Parallel Session I+F2-TA. Invited talk, Impurity effect on irradiation damage evolution in M/HEAs (ID 61). **Keisuke Fukumoto**, Japan.

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Special Session 11:40h, Roth, Markus, Focused Energy canceled

New presentation 11:40h, Volpe, Francesco, Renaissance Fusion, “Renaissance Fusion’s liquid wall and solid materials behind it”

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A.11 99 The effect of Zr addition on oxide particle morphology in ODS-Cu

Toshiki Saito, Hao Yu, Koji Inoue, Zimo Gao, Sosuke Kondo, Yasuyoshi Nagai, Yojiro Oba, Kosuke Hiroi, Ryuta Kasada

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A.52 372 Influence of Thermal Aging on the Deuterium Retention and Permeation Behavior in the RAFM Steel

Wei Wang, Xiang Ji, Chunjing Li

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A.78 546 Testing of Tungsten Powder as a Significant Nuclear Fusion Reactor Waste

Lucie Karásková Nenadálová, Jaroslav Stoklasa

A.79 551 Entrained Hydride Metal-Matrix Composite Shield for Compact Fusion Devices

Jason Trelewicz, David Sprouster, Mirza Shawon, Ethan Peterson, Nesrin Cetiner, Lance Snead



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C.43 538 Estimation of Tritium Residence Time in Two-Phase Lithium Ceramic Based on the Results of Reactor Irradiation Experiments

Zhanna Zaurbekova, Timur Kulsartov, Regina Knitter, Gunta Kizane, Asset Shaimerdenov, Saulet Askerbekov, Magzhan Aitkulov, Yevgen Chikhray, Inesh Kenzhina

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B.22 138 Atomistic modelling of tritium thermodynamics in tungsten and its oxides

Mikael Christensen, Erich Wimmer, Mark Gilbert, Clint Geller, Brian Dron, Duc Nguyen-Manh

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B.43 248 Behavior of deuterium retention in non-damaged and pre-damaged tungsten exposed to deuterium plasma with a rising/declining temperature

Zhen-Hua Ke, Xiu-Li Zhu, Ting Wang, Long Cheng, Yue Yuan, Guang-Hong Lu

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B.85 569 New CuFeTaTiW high entropy alloy for nuclear fusion applications: simulation and experimental production

Marta Dias, R. Martins, A.P. Gonçalves, J.B. Correia, E. Alves

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D.03 89 Tritium trapping assessment in ARC-class reactors breeding blanket

Gabriele Ferrero, Raffaella Testoni, Massimo Zucchetti

D.13 227 A DFT-trained Machine Learning-based interatomic potential for metal foil pump membranes in future nuclear fusion reactors

Christopher Stihl, Pavel Vladimirov

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E.08 218 Crack generation and suppression of W/RAFM steel in hot isostatic pressure diffusion bonding

Yutao Zhai, Zhenya Cao, Ye Zong, Qunying Huang



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F.13 210 Fusion materials studies using JANNuS-Orsay in situ dual ion beam Transmission Electron Microscope

Stéphanie Jublot-Leclerc, Aurelie Gentils

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G.02 119 Evaluation of tensile properties estimated by Small Punch Tests for nuclear structural materials: application of recent standards

Ziwei Li, Dmitry Terentyev, Stefan Holmström, Thomas Pardoen

G.05 163 Comparative study of 8mm small punch discs and Mini-C(T) fracture toughness for EUROFER97 in the ductile-brittle transition region

Henri Wilford Ward, Rebeca Hernandez Pascual, Marta Serrano Garcia, Cory Hamelin, Yiqiang Wang, Mark Jepson, John Echols

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H.03 20 Formation of helical dislocations in FeCr alloys by collision cascades next to screw dislocations

Maria José Caturla, S. Heredia-Avalos, C. D. Denton, J. C. Moreno-Marín, E. Martínez

H.05 33 Effects of Precipitates on Irradiation Resistance and Defect Formation in CoCrFeNi Alloy

Qiu Xu, Zhihong Zhong, K Yasunaga, H Watanabe, Masayuki Tokitani



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ERRATUM - ICFRM-21 BOOK OF ABSTRACTS

Pag. 411, P395

Hydrogen isotope exchange effect on tritium removal in plasma-facing materials of fusion reactors, by Fei Sun et. al.

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